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► To cite this version:

Blaise Faugeras, Jacques Blum, Holger Heumann, Cedric Boulbe. Optimal control of a coupled partial and ordinary differential equations system for the assimilation of polarimetry Stokes vector measurements in tokamak free-boundary equilibrium reconstruction with application to ITER. [Research Report] RR-9014, Inria. 2017. hal-01442462v1

HAL Id: hal-01442462

<https://inria.hal.science/hal-01442462v1>

Submitted on 20 Jan 2017 (v1), last revised 5 Dec 2017 (v2)

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**RESEARCH
REPORT**

N° 9014

January 2017

Project-Teams CASTOR



Optimal control of a coupled partial and ordinary differential equations system for the assimilation of polarimetry Stokes vector measurements in tokamak free-boundary equilibrium reconstruction with application to ITER

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Research Report n° 9014 — January 2017 — 22 pages

Abstract: A method enabling the consistent resolution of the inverse tokamak plasma equilibrium reconstruction problem in the framework of non-linear free-boundary equilibrium coupled to the Stokes model equation for polarimetry is provided. Using optimal control theory we derive the optimality system for this inverse problem. An SQP method is proposed for its numerical resolution. Numerical experiments in the ITER tokamak configuration show that the method is efficient and that the accuracy of the identification of the unknown profile functions is improved compared to the use of classical Faraday measurements.

Key-words: Tokamak equilibrium reconstruction, Stokes model polarimetry, inverse problem, PDE-constrained optimization

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Résumé : Nous proposons une méthode numérique pour la reconstruction de l'équilibre du plasma dans un tokamak en utilisant une modélisation de Stokes pour les données de polarimétrie.

Mots-clés : Reconstruction d'équilibre, tokamak, polarimétrie, modèle de Stokes, problème inverse, EDP, optimisation

1 Introduction

Numerical equilibrium reconstruction is an important and long standing subject in tokamak fusion plasma science [1, 2, 3, 4, 5, 6]. The resolution of this inverse problem: the reconstruction of the poloidal flux function and of the plasma boundary as well as the identification of two non-linear source term functions known as p' and ff' in the Grad-Shafranov equation [7, 8, 9] (see Eq. (6) below), is needed on the one hand for real time control of the plasma during a discharge and on the other hand for post-treatment analysis of equilibrium configurations. The basic set of measurements needed and used are magnetics which provide values of the magnetic poloidal field and flux at several points surrounding the vacuum vessel and the plasma. All free boundary reconstruction codes (e.g. [10, 11, 12, 13, 14, 15, 16, 17]) primarily use these magnetic measurements which proved to be sufficient to identify correctly the plasma boundary and the averaged plasma current density profile [15]. However the difficulty of the reconstruction of the current profile, when only magnetic measurements are used, has been pointed out in [18] and is inherent to the ill-posedness of this inverse problem.

In order to be able to separate more precisely the contributions of the two non-linearities p' and ff' it appears necessary to use supplementary measurements of interferometry and polarimetry which provide integrated quantities along lines of sight or chords crossing the plasma poloidal section [19, 10, 11, 20, 15]. Concerning polarimetry, equilibrium reconstruction codes have until now use Faraday rotation angle measurements only. However in recent papers [21, 22] it has been shown that in modern tokamaks the polarimetry Faraday effect can not be considered alone independently from the Cotton-Mouton effect and that the coupling between them has to be taken into account. This seriously complicates the task of equilibrium reconstruction since it induces to take into account not only the partial differential equation satisfied by the poloidal flux but also its coupling with the Stokes model [23], a system of 3 ordinary differential equations, defined along each polarimetry chords. A first step in this direction is proposed in [24, 25] on simplified plasma models. In this work we provide for the first time a numerical method enabling the consistent resolution of the inverse equilibrium reconstruction problem in the framework of non-linear free-boundary equilibrium coupled to the Stokes model equation.

Next Section 2 is devoted to the formulation of the direct model and the inverse problem. In Section 3 we discuss the numerical methods which we have developed for their resolution and finally in Section 4 some numerical experiments are presented for an ITER configuration.

2 Inverse problem formulation

2.1 Free-boundary plasma equilibrium

The equations which govern the equilibrium of a plasma in the presence of a magnetic field in a tokamak are on the one hand Maxwell's equations satisfied in the whole of space (including the plasma):

$$\nabla \cdot \mathbf{B} = 0, \quad \nabla \times \left(\frac{\mathbf{B}}{\mu} \right) = \mathbf{j}, \quad (1)$$

and on the other hand the equilibrium equation for the plasma itself

$$\nabla p = \mathbf{j} \times \mathbf{B}, \quad (2)$$

where \mathbf{B} is the magnetic field, μ is the magnetic permeability, p is the kinetic pressure and \mathbf{j} is the current density. We refer to standard text books (e.g. [26, 6, 27, 28, 29]) and to [30] for details of the derivation and only state the needed equations in what follows.

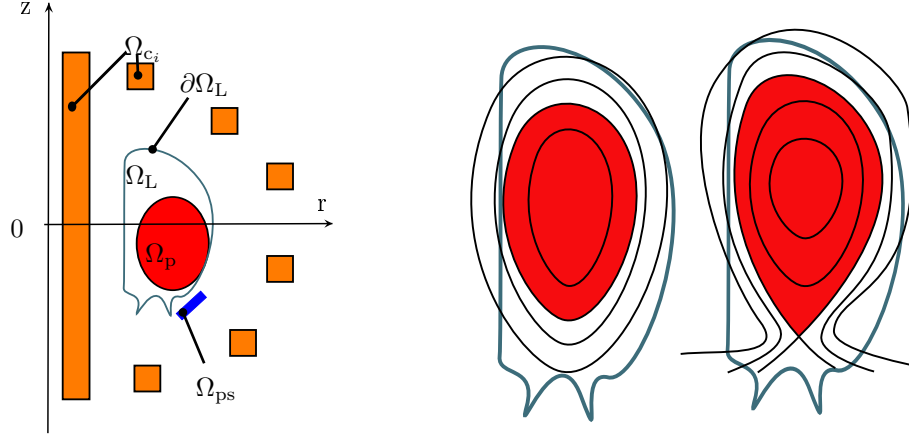


Figure 1: Left: schematic representation of the poloidal plane of a tokamak. Ω_p is the plasma domain, Ω_L is the limiter domain accessible to the plasma, Ω_{c_i} represent poloidal field coils, Ω_{ps} the passive structures. Right: example of a plasma whose boundary is defined by the contact with limiter (left) or by the presence of an X-point (right).

Introducing a cylindrical coordinate system (e_r, e_ϕ, e_z) ($r = 0$ is the major axis of the tokamak torus) and assuming axial symmetry equations (1) and (2) reduce to the following equation for the poloidal flux $\psi(r, z)$ in the poloidal plane $\Omega_\infty = (0, \infty) \times (-\infty, \infty)$:

$$-\Delta^* \psi = j_\phi. \quad (3)$$

where j_ϕ is the toroidal component of \mathbf{j} , and the second order elliptic differential operator Δ^* is defined by

$$\Delta^* := \partial_r \left(\frac{1}{\mu_0 r} \partial_r \cdot \right) + \partial_z \left(\frac{1}{\mu_0 r} \partial_z \cdot \right) := \nabla \cdot \left(\frac{1}{\mu_0 r} \nabla \cdot \right). \quad (4)$$

Here ∇ is the 2D operator in the (r, z) -plane and μ_0 is the magnetic permeability of vacuum (in this work we consider only air-transformer tokamaks such as ITER).

The magnetic field can be decomposed in poloidal and toroidal components

$$\mathbf{B} = \mathbf{B}_p + \mathbf{B}_\phi, \quad \mathbf{B}_p = \frac{1}{r} [\nabla \psi \times e_\phi], \quad \mathbf{B}_\phi = \frac{f}{r} e_\phi. \quad (5)$$

where f is the diamagnetic function. Equation (5) shows that the magnetic surfaces are generated by the rotation of the iso-flux lines around the axis of the torus.

The toroidal component of the current density j_ϕ is zero everywhere outside the plasma domain and the poloidal field coils (and possibly the passive structures). The different sub-domains of the poloidal plane of a schematic tokamak (see Fig. 1) as well as the corresponding expression for j_ϕ are described below:

- Ω_L is the domain accessible to the plasma. Its boundary is the limiter $\partial\Omega_L$.

- Ω_p is the plasma domain where equations (2) and (1) imply that p and f are constant on each magnetic surface i.e. $p = p(\psi)$ and $f = f(\psi)$. One then deduces the so-called Grad-Shafranov equilibrium equation in the plasma [7, 8, 9]

$$-\Delta^* \psi = r p'(\psi) + \frac{1}{\mu_0 r} (f f')(\psi). \quad (6)$$

The right-hand side of (6) is the toroidal component j_ϕ of the plasma current density.

The plasma domain is unknown, i.e. $\Omega_p = \Omega_p(\psi)$, and this is a free boundary problem. This domain is defined by its boundary which is the largest closed ψ iso-contour contained within the limiter Ω_L . The plasma can either be limited if this iso-contour is tangent to the limiter $\partial\Omega_L$ or defined by the presence of an X-point (see Fig. 1).

More precisely

$$\Omega_p(\psi) = \{(r, z) \in \Omega_L, \psi(r, z) \geq \max\left(\max_{(r, z) \in \partial\Omega_L} \psi(r, z), \max_{(r_X, z_X) \in \Omega_L} \psi(r_X, z_X)\right)\}$$

where (r_X, z_X) denotes the coordinates of the saddle points of ψ .

The current density is non-linear in ψ due to the non-linear functions p' and ff' and the definition of the plasma domain $\Omega_p(\psi)$. While $\Omega_p(\psi)$ is fully determined for a given ψ , the two functions p' and ff' are not determined in this modelization. It is the goal of the inverse equilibrium reconstruction problem to determine them. For now let us consider that we are given two functions $A(\psi_N)$ and $B(\psi_N)$ such that

$$j_\phi = \lambda\left(\frac{r}{r_0}A(\psi_N) + \frac{r_0}{r}B(\psi_N)\right) \quad (7)$$

Here r_0 is the major radius of the tokamak vacuum chamber and λ is a scaling coefficient. The normalized poloidal flux $\psi_N(r, z)$ is

$$\psi_N(r, z) = \frac{\psi(r, z) - \psi_a(\psi)}{\psi_b(\psi) - \psi_a(\psi)}. \quad (8)$$

with ψ_a and ψ_b being the flux values at the magnetic axis and at the boundary of the plasma:

$$\begin{aligned} \psi_a(\psi) &:= \psi(r_a(\psi), z_a(\psi)), \\ \psi_b(\psi) &:= \psi(r_b(\psi), z_b(\psi)) \end{aligned} \quad (9)$$

with $(r_a(\psi), z_a(\psi))$ the magnetic axis, where ψ has its global maximum in Ω_L and $(r_b(\psi), z_b(\psi))$ the coordinates of the point that determines the plasma boundary. The point (r_b, z_b) is either an X-point of ψ or the contact point with the limiter $\partial\Omega_L$.

-Domains Ω_{c_i} represent the poloidal field coils carrying currents.

$$j_\phi = \frac{I_i}{S_i} \quad (10)$$

where S_i is the section area of the coil and I_i is a given measured current. Ω_{ps} represents passive structures where the induced current density is assumed to be 0 in this work but can be considered to be measured and given in the same form as Eq. (10)

To sum up, given functions A and B , and currents $\mathbf{I} = \{I_i\}_{i=1}^{N_C}$ in the coils, the free-boundary equilibrium equation for $\psi(r, z)$ on Ω_∞ is the following non-linear boundary value problem

$$\begin{cases} -\Delta^* \psi = \begin{cases} \lambda\left(\frac{r}{r_0}A(\psi_N) + \frac{r_0}{r}B(\psi_N)\right) & \text{in } \Omega_p(\psi) \\ \frac{I_i}{S_i} & \text{in } \Omega_{c_i} \\ 0 & \text{elsewhere} \end{cases} \\ \psi(0, z, t) = 0 \\ \lim_{\|(r, z)\| \rightarrow +\infty} \psi(r, z) = 0 \end{cases} \quad (11)$$

This formulation on an infinite domain is not used directly in computations where we use finite elements on a truncated bounded domain. The infinite domain is reduced to a semi circular computational domain by an uncoupling procedure [31, 32]. We chose a semi-circle Γ of radius ρ_Γ surrounding the coil domains Ω_{c_i} (see Fig. 2) and define the computation domain Ω having boundary $\partial\Omega = \Gamma \cup \Gamma_0$, where $\Gamma_0 = \{(0, z), z \in [-\rho_\Gamma, \rho_\Gamma]\}$. The weak formulation of the equilibrium problem on which the finite element method relies uses the following function space

$$V := \left\{ \psi : \Omega \rightarrow \mathbb{R}, \|\psi\| < \infty, \left\| \frac{|\nabla\psi|}{r} \right\| < \infty, \psi|_{\Gamma_0} = 0 \right\} \cap C^0(\bar{\Omega}),$$

with

$$\|\psi\|^2 = \int_{\Omega} \psi^2 r \, dr dz.$$

and can be written as:

Given function A and B , and currents \mathbf{I} , find $\psi \in V$ such that for all $\xi \in V$

$$\mathbf{a}(\psi, \xi) - \mathbf{J}_p(\psi, \xi; A, B) + \mathbf{c}(\psi, \xi) = \ell(\mathbf{I}, \xi), \quad (12)$$

where

$$\begin{aligned} \mathbf{a}(\psi, \xi) &:= \int_{\Omega} \frac{1}{\mu_0 r} \nabla\psi \cdot \nabla\xi \, dr dz, \\ \mathbf{J}_p(\psi, \xi; A, B) &:= \int_{\Omega_p(\psi)} \lambda \left(\frac{r}{r_0} A(\psi_N) + \frac{r_0}{r} B(\psi_N) \right) \xi \, dr dz, \\ \ell(\mathbf{I}, \xi) &:= \sum_{i=1}^{N_C} \frac{I_i}{S_i} \int_{\Omega_{c_i}} \xi \, dr dz, \end{aligned} \quad (13)$$

and the bilinear form $\mathbf{c} : V \times V \rightarrow \mathbb{R}$ is accounting for the boundary conditions at infinity. We refer to [30] for its precise expression and to [33, Chapter 2.4] for the details on its the derivation.

2.2 Stokes model for polarimetry

Polarimetry consists in measurements of the change of state of polarization of an electromagnetic radiation propagating across the magnetized plasma along several chords distributed on the poloidal section of the tokamak. One method of describing the state of polarization is to introduce a Stokes vector $\mathbf{s} = (s_1, s_2, s_3)$. The evolution of the polarization when the laser beam crosses the plasma is then given by the following Stokes equation on each chord:

$$\begin{cases} \frac{d\mathbf{s}}{dZ} = \mathbf{G}\mathbf{s}, & \text{on } (Z_0, Z_1], \\ \mathbf{s}(Z_0) = \mathbf{s}_0 \end{cases} \quad (14)$$

We refer to [23] for details on this modelization. Here we have introduced a coordinate system $(\mathbf{e}_X, \mathbf{e}_Y, \mathbf{e}_Z)$ attached to a chord C . Z is the coordinate tangent to the chord, X represents the toroidal direction and Y the direction perpendicular to Z in the poloidal plane. In this coordinate system the components of the magnetic field are denoted by (B_X, B_Y, B_Z) .

The initial polarization is given by \mathbf{s}_0 at Z_0 . Z_1 corresponds to the location of the output measurement sensor. The 3×3 matrix \mathbf{G} is such that $\mathbf{G}\mathbf{s} = \boldsymbol{\Omega} \times \mathbf{s}$ where vector $\boldsymbol{\Omega} = (\Omega_1, \Omega_2, \Omega_3)$ has components

$$\Omega_1 = C_1 N_e (B_X^2 - B_Y^2), \quad \Omega_2 = 2C_1 N_e B_X B_Y, \quad \Omega_3 = C_3 N_e B_Z \quad (15)$$

Here the electronic density in the plasma, $N_e = N_e(\psi_N)$, is assumed to be constant on the flux surfaces. Ω depends on N_e as well as on ψ and on the function B from Eq. (7). Indeed from Eq. (5) we have that the components of the magnetic field can be written as

$$B_Z = -\frac{1}{r}\nabla\psi \cdot \mathbf{e}_Y, \quad B_Y = \frac{1}{r}\nabla\psi \cdot \mathbf{e}_Z, \quad B_X = \frac{f}{r} \quad (16)$$

where function f is related to function B through the relation $ff' = \lambda\mu_0 r_0 B$. We denote these dependencies by $\mathbf{G}(\psi, B, N_e)$. Constants C_1 and C_3 depend on the wavelength of the beam radiation.

Hence in order to use polarimetry measurements with Stokes modelization in addition to basic magnetic measurements for the identification of functions A and B one has to supplement equation (11) or (12) with, for each line of sight, a system of linear ordinary differential equations (14) for the Stokes vector.

2.3 The inverse identification problem

Magnetics constitute the basic set of experimental measurements used in equilibrium reconstruction for the identification of functions A and B . They consist in measurements of projections of the poloidal magnetic field, $\mathbf{B}_p \cdot \mathbf{d}$ at several locations around the vacuum vessel of the tokamak (unit vector \mathbf{d} varies with each B-probe) and of measurements of ψ obtained from flux loops at several locations too (see Fig. 3).

In order to be able to use polarimetric measurements the electronic density function, $N_e(\psi_N)$ has to be known. It is therefore also going to be identified using interferometric measurements which give the density line integrals over each of the N_L chords C^i , $i = 1, \dots, N_L$:

$$N_{e,obs}^i \approx \int_{C^i} N_e(\psi_N) dZ^i$$

Polarimetric measurements as they are considered in all former equilibrium reconstruction (e.g. [19, 20, 15]) studies give the Faraday rotation of the angle of the infrared radiation crossing the section of the plasma along the different chords

$$\alpha_{obs}^i \approx \frac{1}{2} \int_{C^i} \Omega_3 dZ^i \quad (17)$$

As detailed in [23] this is an approximation to one component of the Stokes vector $\mathbf{s}(Z_1^i)$ only valid for small Faraday and Cotton-Mouton effects. On the contrary in this study we consider that polarimetric measurements are given by the full Stokes vector at the Z_1^i coordinate on each chord C^i

$$\mathbf{s}_{obs}^i \approx \mathbf{s}(Z_1^i)$$

Indeed it is stated in [22, Section 2] and [23, Section 8] that the components of the Stokes vector are related directly to quantities measured by the polarimetric system.

At this point we have defined a direct model given by the equilibrium equation (12) and Stokes equation (14) on every chord, control variables, A , B and N_e and measurements to which are attached experimental errors represented by the standard deviations σ_s in Eq. (19) below. The identification problem can now be formulated as a constrained minimization problem for the following cost function ($\{\mathbf{s}\}$ denotes the vector $(\mathbf{s}^1, \dots, \mathbf{s}^{N_L})$ of Stokes vectors for all chords):

$$J(\psi, \{\mathbf{s}\}, A, B, N_e) = J_{obs}(\psi, \{\mathbf{s}\}, N_e) + R(A, B, N_e) \quad (18)$$

where the least-square term is

$$\begin{aligned}
J_{obs}(\psi, \{\mathbf{s}\}, N_e) = & \\
& \sum_{i=1}^{N_B} \frac{1}{2\sigma_{B_i}^2} ((\mathbf{B}_p(r_i, z_i) \cdot \mathbf{d}_i) - B_{p,obs}^i)^2 + \sum_{i=1}^{N_F} \frac{1}{2\sigma_{F_i}^2} (\psi(r_i, z_i) - \psi_{obs}^i)^2 \\
& + \sum_{i=1}^{N_L} \frac{1}{2\sigma_{N_i}^2} \left(\int_{C_i} N_e(\psi_N) dZ - N_{e,obs}^i \right)^2 + \sum_{i=1}^{N_L} \frac{1}{2\sigma_{S_i}^2} \|\mathbf{s}^i(Z_1^i) - \mathbf{s}_{obs}^i\|^2
\end{aligned} \tag{19}$$

and the regularization term is

$$\begin{aligned}
R(A, B, N_e) = & \\
& \frac{\varepsilon_A}{2} \int_0^1 [A''(x)]^2 dx + \frac{\varepsilon_B}{2} \int_0^1 [B''(x)]^2 dx + \frac{\varepsilon_{N_e}}{2} \int_0^1 [N_e''(x)]^2 dx
\end{aligned} \tag{20}$$

under the constraint of the model equations (21) and (22) written again below:

$$\mathbf{a}(\psi, \xi) - \mathbf{J}_p(\psi, \xi; A, B) + \mathbf{c}(\psi, \xi) = \ell(\mathbf{I}, \xi), \quad \forall \xi \in V \tag{21}$$

and for all chords C^i , $i = 1, \dots, N_L$:

$$\begin{cases} \frac{d\mathbf{s}^i}{dZ^i} = \mathbf{G}(\psi, B, N_e)\mathbf{s}^i, & \text{on } (Z_0^i, Z_1^i], \\ \mathbf{s}^i(Z_0^i) = \mathbf{s}_0^i \end{cases} \tag{22}$$

The unknown functions A , B and N_e are supposed to belong to a set \mathcal{U} of regular functions defined on $[0, 1]$ and such that $A(1) = B(1) = N_e(1) = 0$.

Cost function (18) is not quadratic and the constraints are non-linear. In order to formulate the first order optimality conditions we introduce Lagrange multipliers p and $\{\mathbf{q}\}$ associated respectively to constraint (21) and (22), and formulate the Lagrangian for the optimization problem:

$$\begin{aligned}
\mathcal{L}(\psi, \{\mathbf{s}\}, A, B, N_e, p, \{\mathbf{q}\}) = & J(\psi, \{\mathbf{s}\}, A, B, N_e) \\
& + \mathbf{a}(\psi, p) - \mathbf{J}_p(\psi, p; A, B) + \mathbf{c}(\psi, p) - \ell(\mathbf{I}, p) \\
& + \sum_{i=1}^{N_L} \int_{Z_0^i}^{Z_1^i} \left(\left(\frac{d\mathbf{s}^i}{dZ^i} - \mathbf{G}(\psi, B, N_e)\mathbf{s}^i \right) \cdot \mathbf{q}^i \right) dZ^i
\end{aligned} \tag{23}$$

A solution to the optimization problem is a stationary point of this Lagrangian. Let us formally derive the optimality system. The nullity of the ψ derivative of \mathcal{L} in direction h_ψ yields the weak formulation of adjoint equilibrium problem for p :

$$\begin{aligned}
\mathbf{a}(h_\psi, p) - D_\psi \mathbf{J}_p(\psi, p; A, B)(h_\psi) + \mathbf{c}(h_\psi, p) = & -D_\psi J_{obs}(\psi, \{\mathbf{s}\}, N_e)(h_\psi) \\
& + \sum_{i=1}^{N_L} \int_{Z_0^i}^{Z_1^i} \left((D_\psi [\mathbf{G}(\psi, B, N_e)\mathbf{s}^i](h_\psi)) \cdot \mathbf{q}^i \right) dZ^i \quad \forall h_\psi \in V
\end{aligned} \tag{24}$$

This equation involves a non-usual second term on the right hand side coming from the coupling between (21) and (22). The computation of the ψ derivative of \mathbf{J}_p is not straightforward because of the dependence of the integration domain Ω_p on ψ . We refer to [6, 30] for this point.

The nullity of the \mathbf{s}^i derivative of \mathcal{L} yields the adjoint Stokes equations for \mathbf{q}^i on each chord. They have to be integrated backwards from a final condition at Z_1^i :

$$\begin{cases} -\frac{d\mathbf{q}^i}{dZ^i} - \mathbf{G}(\psi, B, n)^T \mathbf{q}^i = 0, \\ \mathbf{q}^i(Z_1^i) = -(\mathbf{s}^i(Z_1^i) - \mathbf{s}_{obs}^i) \end{cases} \tag{25}$$

The A , B and N_e derivatives of \mathcal{L} yield the gradient of cost function (18) the constraint equations being satisfied:

$$\varepsilon_A \int_0^1 A''(x) h_A''(x) dx - \int_{\Omega_p(\psi)} \lambda \frac{r}{r_0} h_A(\psi_N) p dr dz = 0, \quad \forall h_A \in \mathcal{U} \quad (26)$$

$$\begin{aligned} \varepsilon_B \int_0^1 B''(x) h_B''(x) dx - \int_{\Omega_p(\psi)} \lambda \frac{r_0}{r} h_B(\psi_N) p dr dz \\ - \sum_{i=1}^{N_L} \int_{Z_0^i}^{Z_1^i} ((D_B [\mathbf{G}(\psi, B, N_e) \mathbf{s}^i] (h_B)) \cdot \mathbf{q}^i) dZ^i = 0, \quad \forall h_B \in \mathcal{U} \end{aligned} \quad (27)$$

$$\begin{aligned} \varepsilon_{N_e} \int_0^1 N_e''(x) h_{N_e}''(x) dx + D_{N_e} J_{obs}(\psi, \{\mathbf{s}\}, N_e)(h_{N_e}) \\ - \sum_{i=1}^{N_L} \int_{Z_0^i}^{Z_1^i} ((D_{N_e} [\mathbf{G}(\psi, B, N_e) \mathbf{s}^i] (h_{N_e})) \cdot \mathbf{q}^i) dZ^i = 0, \quad \forall h_{N_e} \in \mathcal{U} \end{aligned} \quad (28)$$

As usual the derivative of \mathcal{L} with respect to p and \mathbf{q}^i gives back the model equations (21) and (22) respectively. Equations (24)-(25), (26)-(28) and (21)-(22) form the optimality system for the optimization problem under consideration. In the next section we discretize it and propose a numerical algorithm for its resolution.

3 Numerical methods

3.1 Discretization of the direct model

Equilibrium equation (21) is discretized using a P1 finite element method based on triangular meshes [34, 35, 30]. From now on let us also assume that functions A and B are decomposed in a basis of functions ϕ_i defined on $[0, 1]$. We use cubic spline functions in this work.

$$A(x) = \sum_{i=1}^N u_{A_i} \phi_i(x) \quad B(x) = \sum_{i=1}^N u_{B_i} \phi_i(x) \quad (29)$$

and let us denote $\mathbf{u} = (\mathbf{u}_A, \mathbf{u}_B)$ the vector of degrees of freedom of A and B in the decomposition basis. Classically approximating ψ by $\psi_h = \sum_{i=1}^{N_\psi} \psi_i \lambda_i(r, z)$ on the finite element approximation space as well as the operators of (21) and taking all basis elements λ_i as test functions lead to the following non-linear system of N_ψ equations:

$$(\mathbf{A} + \mathbf{C})\boldsymbol{\psi} - \mathbf{J}_p(\boldsymbol{\psi}, \mathbf{u}) - \mathbf{L}\mathbf{I} = 0 \quad (30)$$

where $\boldsymbol{\psi}$ denotes the vector of finite element coefficients $\{\psi_i\}_{i=1}^{N_\psi}$ and other notations are obvious.

In order to solve direct equilibrium problem (30), \mathbf{u} and \mathbf{I} being given, Newton method is used. As for the continuous case the computation of the derivative $D_{\boldsymbol{\psi}} \mathbf{J}_p(\boldsymbol{\psi}, \mathbf{u})(h_\psi)$ has to be conducted with care. In this work we use the derivatives of the discrete operator and refer to [30, section 3.2] and [30, section 3.3] for technical details.

Let us now turn to the discretization of the Stokes model for polarimetry. We use a Crank-Nicolson scheme for the integration of (22). Each chord C^i is discretized with N^i points $\mathbf{x}^{i,k+1} = \mathbf{x}^{i,k} + h^{i,k} \mathbf{t}^i$ where $k = 1 \dots N^i - 1$, $\mathbf{x} = (r, z)$ and \mathbf{t}^i is a unit vector tangent to the chord. The

first point $\mathbf{x}^{i,1}$ corresponds to $(0, 0, Z_0^i)$ in the coordinate system attached to chord i and the last one \mathbf{x}^{i,N^i} to $(0, 0, Z_1^i)$. Although the discretization steps $h^{i,k}$ are chosen of the same order of magnitude as h , the discretization parameter for the finite element method, the $x^{i,k}$ points are independent of the triangular mesh. Let us assume that similarly to functions A and B , the density N_e is decomposed in a function basis with degrees of freedom \mathbf{v} then with obvious notations the integration scheme for \mathbf{s}^i on chord i can be written as:

$$\begin{aligned} \mathbf{s}^{i,1} &= \mathbf{s}_0^i \\ \left[\mathbf{I}_3 - \frac{h^{i,k}}{2} \mathbf{G}(\psi_h(\mathbf{x}^{i,k+1}), \mathbf{u}, \mathbf{v}) \right] \mathbf{s}^{i,k+1} \\ - \left[\mathbf{I}_3 + \frac{h^{i,k}}{2} \mathbf{G}(\psi_h(\mathbf{x}^{i,k}), \mathbf{u}, \mathbf{v}) \right] \mathbf{s}^{i,k} &= 0 \quad k = 1 \dots N^i - 1 \end{aligned} \quad (31)$$

which can be put into the more concise

$$\mathbf{M}^i(\boldsymbol{\psi}, \mathbf{u}, \mathbf{v}) \mathbf{S}^i - \mathbf{S}_0^i = 0 \quad (32)$$

where \mathbf{M}^i is a $3N^i \times 3N^i$ band diagonal matrix, $\mathbf{S}^i = (\mathbf{s}^{i,1}, \dots, \mathbf{s}^{i,N^i})$ is the vector of all Stokes vector states along chord i and $\mathbf{S}_0^i = (\mathbf{s}_0^i, 0, \dots, 0)$ represents the initial conditions.

3.2 The discrete identification problem

Using the discrete variables of the preceding section cost function (18) can also be discretized leading to the following expression

$$\begin{aligned} J(\boldsymbol{\psi}, \{\mathbf{S}\}, \mathbf{u}, \mathbf{v}) &= \frac{1}{2} \|\mathbf{H}\boldsymbol{\psi} - \mathbf{m}\|^2 \\ &+ \frac{1}{2} \|\mathbf{W}(\boldsymbol{\psi})\mathbf{v} - \mathbf{N}_{e,obs}\|^2 + \sum_{i=1}^{N_L} \frac{1}{2} \|\mathbf{E}\mathbf{S}^i - \mathbf{s}_{obs}^i\|^2 \\ &+ \frac{1}{2} \|\mathbf{R}_u \mathbf{u}\|^2 + \frac{1}{2} \|\mathbf{R}_v \mathbf{v}\|^2 \end{aligned} \quad (33)$$

Here $\{\mathbf{S}\}$ is the vector $(\mathbf{S}^1, \dots, \mathbf{S}^{N_L})$. In order to lighten notations the $\frac{1}{\sigma}$ terms have been dropped and are assumed to be included in the observation operators and in the measurements. The linear observation operator \mathbf{H} maps the finite element approximation $\boldsymbol{\psi}$ to the equivalent of magnetic measurements \mathbf{m} . The non-linear observation operator $\mathbf{W}(\boldsymbol{\psi})\mathbf{v}$ represents the numerical quadrature of the electronic density over the different chords. This term is linear in \mathbf{v} but not in $\boldsymbol{\psi}$. The observation operator for Stokes vectors is given by matrix \mathbf{E} such that $\mathbf{E}\mathbf{S}^i = \mathbf{s}^{i,N^i}$ is the Stokes vector at the observation point. The last two terms involving matrices \mathbf{R}_u and \mathbf{R}_v are the discretization of the regularization terms in which we have gathered the contributions from functions A and B .

The discrete identification problem can now be stated as

$$\min_{\boldsymbol{\psi}, \{\mathbf{S}\}, \mathbf{u}, \mathbf{v}} J(\boldsymbol{\psi}, \{\mathbf{S}\}, \mathbf{u}, \mathbf{v}) \quad (34)$$

subject to the constraint of the model

$$(\mathbf{A} + \mathbf{C})\boldsymbol{\psi} - \mathbf{J}_p(\boldsymbol{\psi}, \mathbf{u}) - \mathbf{L}\mathbf{I} = 0 \quad (35)$$

and

$$\mathbf{M}^i(\boldsymbol{\psi}, \mathbf{u}, \mathbf{v}) \mathbf{S}^i - \mathbf{S}_0^i = 0, \quad i = 1, \dots, N^L \quad (36)$$

Similarly to the continuous case we introduce Lagrange multipliers $\mathbf{p} \in \mathbb{R}^{N_\psi}$ and $\mathbf{q}^i \in \mathbb{R}^{3N^i}$ to formulate the Lagrangian for the optimization problem (34)-(36):

$$\begin{aligned} \mathcal{L}(\boldsymbol{\psi}, \{\mathbf{S}\}, \mathbf{u}, \mathbf{v}, \mathbf{p}, \{\mathbf{q}\}) &= J(\boldsymbol{\psi}, \{\mathbf{S}\}, \mathbf{u}, \mathbf{v}) \\ &+ (\mathbf{p} \cdot ((\mathbf{A} + \mathbf{C})\boldsymbol{\psi} - \mathbf{J}_p(\boldsymbol{\psi}, \mathbf{u}) - \mathbf{L}\mathbf{I})) + \sum_{i=1}^{N_L} (\mathbf{q}^i \cdot (\mathbf{M}^i(\boldsymbol{\psi}, \mathbf{u}, \mathbf{v})\mathbf{S}^i - \mathbf{S}_0^i)) \end{aligned} \quad (37)$$

and compute the discrete optimality system:

$$\begin{aligned} \mathbf{H}^T(\mathbf{H}\boldsymbol{\psi} - \mathbf{m}) + D_\psi [\mathbf{W}(\boldsymbol{\psi})\mathbf{v}]^T (\mathbf{W}(\boldsymbol{\psi})\mathbf{v} - \mathbf{N}_{e,obs}) \\ + \sum_{i=1}^{N_L} D_\psi [\mathbf{M}^i(\boldsymbol{\psi}, \mathbf{u}, \mathbf{v})\mathbf{S}^i]^T \mathbf{q}^i + (\mathbf{A} + \mathbf{C} - D_\psi \mathbf{J}_p(\boldsymbol{\psi}, \mathbf{u}))^T \mathbf{p} = 0 \end{aligned} \quad (38)$$

$$\mathbf{E}^T(\mathbf{E}\mathbf{S}^i - \mathbf{s}_{obs}^i) + \mathbf{M}^i(\boldsymbol{\psi}, \mathbf{u}, \mathbf{v})^T \mathbf{q}^i = 0, \quad i = 1, \dots, N_L \quad (39)$$

$$\mathbf{R}_u^T \mathbf{R}_u \mathbf{u} - D_u \mathbf{J}_p(\boldsymbol{\psi}, \mathbf{u})^T \mathbf{p} + \sum_{i=1}^{N_L} D_u [\mathbf{M}^i(\boldsymbol{\psi}, \mathbf{u}, \mathbf{v})\mathbf{S}^i]^T \mathbf{q}^i = 0 \quad (40)$$

$$\mathbf{R}_v^T \mathbf{R}_v \mathbf{v} + \mathbf{W}(\boldsymbol{\psi})^T (\mathbf{W}(\boldsymbol{\psi})\mathbf{v} - \mathbf{N}_{e,obs}) + \sum_{i=1}^{N_L} D_v [\mathbf{M}^i(\boldsymbol{\psi}, \mathbf{u}, \mathbf{v})\mathbf{S}^i]^T \mathbf{q}^i = 0 \quad (41)$$

$$(\mathbf{A} + \mathbf{C})\boldsymbol{\psi} - \mathbf{J}_p(\boldsymbol{\psi}, \mathbf{u}) - \mathbf{L}\mathbf{I} = 0 \quad (42)$$

$$\mathbf{M}^i(\boldsymbol{\psi}, \mathbf{u}, \mathbf{v})\mathbf{S}^i - \mathbf{S}_0^i = 0, \quad i = 1, \dots, N^L \quad (43)$$

Equation (38) is the discrete adjoint equilibrium problem for \mathbf{p} . Equation (39) is the adjoint Stokes equation on each chord for \mathbf{q}^i . The transposed operator corresponds to the backwards integration from Z_1^i to Z_0^i of the adjoint Stokes equation. Equations (40) and (41) give the gradient of the cost function seen as a function of (\mathbf{u}, \mathbf{v}) only, when the model constraint is satisfied. The last two equations are the model constraint.

The optimality system (38)-(43) is a system of non-linear equations $\mathbf{F}(\mathbf{X}) = 0$ with $\mathbf{X} = (\boldsymbol{\psi}, \{\mathbf{S}\}, \mathbf{u}, \mathbf{v}, \mathbf{p}, \{\mathbf{q}\})$. We solve it thanks to a Newton-type method. If the second order derivative terms are neglected in the computation of the Jacobian of the optimality system, $\mathbf{K}(\mathbf{X}) \approx D_{\mathbf{X}}\mathbf{F}(\mathbf{X})$, the following approximated Newton iteration is obtained

$$\mathbf{K}(\mathbf{X}^k)(\mathbf{X}^{k+1} - \mathbf{X}^k) = -\mathbf{F}(\mathbf{X}^k) \quad (44)$$

where

$$\mathbf{K} = \begin{bmatrix} \mathbf{H}^T \mathbf{H} & & & & \mathbf{A}^T & & \\ +D_\psi [\mathbf{W}\mathbf{v}]^T & 0 & 0 & D_\psi [\mathbf{W}\mathbf{v}]^T & +\mathbf{C}^T & D_\psi [\mathbf{M}\mathbf{S}]^T \\ \times D_\psi [\mathbf{W}\mathbf{v}] & & & \times \mathbf{W} & -D_\psi \mathbf{J}_p^T & \\ 0 & \mathbf{E}^T \mathbf{E} & 0 & 0 & 0 & \mathbf{M}^T \\ 0 & 0 & \mathbf{R}_u^T \mathbf{R}_u & 0 & -D_u \mathbf{J}_p^T & D_u [\mathbf{M}\mathbf{S}]^T \\ \mathbf{W}^T & & & \mathbf{R}_v^T \mathbf{R}_v & 0 & D_v [\mathbf{M}\mathbf{S}]^T \\ \times D_\psi [\mathbf{W}\mathbf{v}] & 0 & 0 & +\mathbf{W}^T \mathbf{W} & & \\ \mathbf{A} & & & & & \\ +\mathbf{C} & 0 & -D_u \mathbf{J}_p & 0 & 0 & 0 \\ -D_\psi \mathbf{J}_p & & & & & \\ D_\psi [\mathbf{M}\mathbf{S}] & \mathbf{M} & D_u [\mathbf{M}\mathbf{S}] & D_v [\mathbf{M}\mathbf{S}] & 0 & 0 \end{bmatrix} \quad (45)$$

Here all the dependances in $\boldsymbol{\psi}$, \mathbf{u} or \mathbf{v} have been dropped to lighten notations and we have defined

$$\mathbf{M} = \begin{bmatrix} \mathbf{M}^1 & 0 & \dots & 0 \\ & \ddots & & \\ & & \ddots & \\ 0 & \dots & 0 & \mathbf{M}^{N_L} \end{bmatrix} \quad (46)$$

as well as

$$D_\psi[\mathbf{MS}] = \begin{bmatrix} D_\psi[\mathbf{M}^1 \mathbf{S}^1] \\ \vdots \\ D_\psi[\mathbf{M}^{N_L} \mathbf{S}^1] \end{bmatrix} \quad (47)$$

and $D_{\mathbf{u}}[\mathbf{MS}]$ and $D_{\mathbf{v}}[\mathbf{MS}]$ are similarly defined.

The approximated Newton iterative scheme (44) presented here is the adaptation to the particular case of tokamak equilibrium reconstruction using magnetic and Stokes vector measurements, of PDE-constrained optimization methods [36] and falls into the family of sequential quadratic programming (SQP) methods [37]. The algorithm presented here can be modified to consider the cases where only magnetic measurements or polarimetry measurements using classical formulation (17) are used.

One approximated Newton iteration for the optimality system corresponds exactly to the minimization of a quadratic functional (J in Eq. (33) in which the $\mathbf{W}(\boldsymbol{\psi})\mathbf{v}$ term is linearized at $(\boldsymbol{\psi}^k, \mathbf{v}^k)$) under linear constraints (model equations (35) and (36) linearized at $(\boldsymbol{\psi}^k, \{\mathbf{S}\}^k, \mathbf{u}^k, \mathbf{v}^k)$). This is the point of view adopted in [6, 20] where at each iteration (called external iteration) the linear quadratic control problem can be solved by a sequence of internal conjugate gradient iterations. This approach is useful if system (44) becomes too large for the linear solver (e.g. in case of very small discretization steps). However with modern computers, performant linear solver libraries and reasonable discretization parameters this resolution is possible and fast. The numerical results presented in the next section are obtained with the iterative scheme (44) directly solving the optimality system.

Similarly to Newton method used for the resolution of the direct equilibrium problem of Eq. (30), the performance of the SQP method used for the resolution of the identification problem relies on the accuracy of the derivative terms $D_\psi \mathbf{J}_p$, $D_{\mathbf{u}} \mathbf{J}_p$, $D_\psi[\mathbf{MS}]$, $D_{\mathbf{u}}[\mathbf{MS}]$ and $D_{\mathbf{v}}[\mathbf{MS}]$. In this work we have implemented the exact derivatives of the fully discretized operators. This essential but very technical work is not further detailed here.

4 Numerical experiments

The numerical methods presented in the previous section have been implemented in the code FEEQS.M which is a Matlab implementation of the methods for free-boundary equilibrium computations presented in [30]. The ITER geometry and magnetic sensors (145 B probes and 4 flux loops) positions are taken from the European Integrated Tokamak Modeling database [38, 39]. The 15 polarimetry viewing chords are taken from [40].

The general methodology is the following. Synthetic measurements are generated integrating the direct equilibrium and Stokes models given reference functions A , B and N_e . Figures 2 and 3 show an example of such an equilibrium as well as the magnetic sensors and the polarimetry chords. In all simulations the scaling factor λ in the current density (7) is computed such that the total plasma current is $I_p = 15 \times 10^6$ [A]. The vacuum toroidal field is $B_0 = 3.866$ [T] and $r_0 = 6.2$ [m].

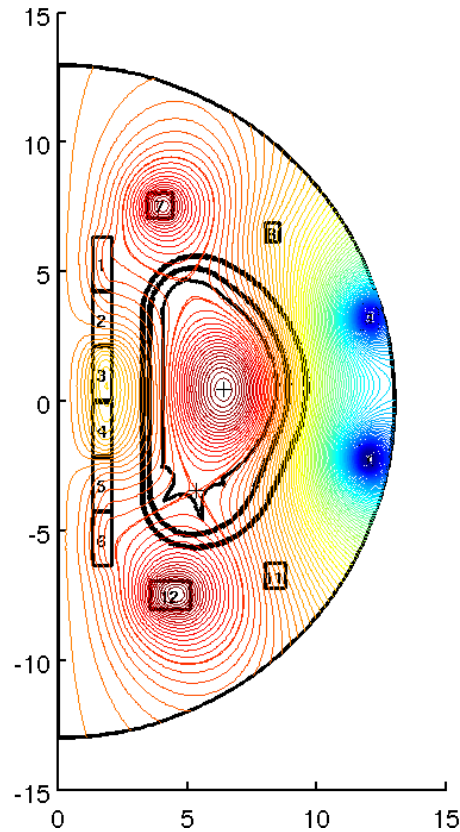


Figure 2: Computed ITER equilibrium for TC1. Isoflux contours are shown on the full computation domain. The poloidal field coils appear numbered from 1 to 12. The vacuum vessel and limiter contours are also shown.

A wavelength of $l = 0.195$ [mm] is assumed for the laser beams and the associated values for constants appearing in Stokes equation are $C_1 = 2.42 \times 10^{-20} l^3 = 1.7944 \times 10^{-22}$ [$m^2 T^{-2}$] and $C_3 = 5.23 \times 10^{-19} l^2 = 1.9887 \times 10^{-20}$ [$m^2 T^{-1}$]. Following [22] the initial Stokes vectors are chosen as $\mathbf{s}_0^i = (0, 1, 0)$.

Then in a second step these measurements are plugged in cost function (33) and the optimization problem is solved using the iterative algorithm (44) presented above. The initial guess for this resolution consists in a given circular plasma boundary in which the flux ψ is a constant and outside of which it is 0, as well as affine functions $A(x) = B(x) = 1 - x$. A first Newton iteration solely for the equilibrium problem with fixed I_p [30] is performed in order to compute a first ψ map and to give a value to the scaling factor λ . Thereon this factor is kept fixed and for what concerns the plasma current density only \mathbf{u} representing the functions to be identified evolve during the iterations. Convergence is assumed when

$$\|\mathbf{X}^{k+1} - \mathbf{X}^k\| / \|\mathbf{X}^k\| < 10^{-12}$$

which takes about 10 iterations to be satisfied.

Three types of experiments are conducted: type M in which only magnetics are used; type MF in which magnetics, interferometry and classical Faraday measurements are used; and type MS in which magnetics, interferometry and Stokes vector measurements are used.

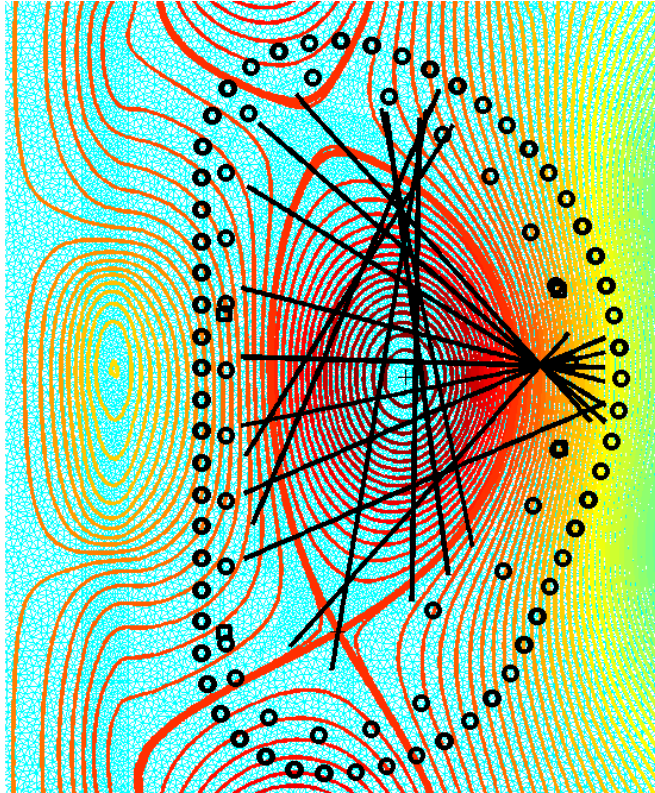


Figure 3: Zoom on the vacuum vessel for computed ITER equilibrium TC1. Isoflux contours are shown, the thick one corresponds to the value ψ_b of the plasma boundary defined by the presence of an X-point. The finite element mesh is visible in the background. The small black circles indicate the position of the 145 B probes. The 4 small squares indicate the position of the 4 flux loops. The black segments represent the 15 chords used for intereferometry and polarimetry computations.

		B probes	flux loops	interfero.	polarimetry		
M	TC1	0.1855	0.1828				
	TC2	0.0844	0.3357				
MF	TC1	0.1721	0.1682	1.8643	5.5860		
	TC2	0.0790	0.3212	4.0387	4.1222		
MS	TC1	0.1679	0.1664	2.2076	5.2949	1.1283	2.4305
	TC2	0.0804	0.3219	3.3977	3.3746	0.6373	3.2967

Table 1: Mean relative errors ($\times 10^2$) between real value of measured quantities (i.e used measurements without noise) and reconstructed values for TC1 and TC2, with noisy measurements (1% on magnetics and 10% on interferometry and polarimetry).

The finite element mesh is composed of $N_\psi = 30449$ nodes among which 30270 correspond to free values of ψ (the remaining correspond to the imposed boundary condition $\psi = 0$ on the axis $r = 0$). The discretization step on each chord is chosen to be $h = 0.05$ [m] giving a vector $\{\mathbf{S}\}$ of size 3×2022 . Each function to be identified is decomposed in 11 splines defined on $[0, 1]$ with knots at $0, 0.1, \dots, 1$. Therefore X is a vector of size $2 \times 30270 + 2 \times 11 = 60562$ for type M experiments, $2 \times 30270 + 3 \times 11 = 60573$ for type MF and $2 \times (30270 + 3 \times 2022) + 3 \times 11 = 72705$ for type MS experiments.

We consider 2 test cases (TC). In TC1, the reference functions are taken as $A(x) = B(x) = (1 - x^{3/2})^2$ and in TC2 we choose $A(x) = (1 - x)^{3/2}$ and $B(x) = 1 - x^2$. In both cases $N_e(x) = N_{e0}(1 - x^3)$ with $N_{e0} = 10^{19}$ [m^{-3}].

For each numerical experiment the regularization parameters ε and ε_{N_e} are tuned to their lowest value, typically $10^{-3} - 10^{-2}$, avoiding oscillations in the reconstructed profiles or non convergence of the code.

In order to validate the good convergence properties of the code we first consider the case of perfect measurements (without additional noise). In all experiments the relative errors between measurements and reconstructed values is of order 10^{-5} for magnetics and 10^{-4} for interferometry and polarimetry indicating an excellent fit. The relative error on ψ is also very small and the plasma boundary is perfectly recovered. The reconstructed profiles are shown on Fig. 4. The N_e profile is very well recovered. Profiles A and B are also well recovered in the 3 experiments M, MF and MS even though the discrepancies between the real reference profile values and the reconstructed ones near $\psi_N = 0$ (the magnetic axis) tend to be larger than close to $\psi_N = 1$ (the plasma boundary). This is typical of this plasma current identification problem [15]. The main point here is that, as can be seen from Fig. 4, the use of Faraday measurements (MF) or of Stokes vector measurements (MS) decreases this error. In this no noise experiment the difference between MF and MS is however barely distinguishable. It is clearer when noise is added.

We add a 1% noise on magnetics and a 10% noise on interferometry and polarimetry measurements and conduct a new set of identification experiments. Once again the resulting relative errors on ψ are very small and the plasma boundary is perfectly recovered for both TC1 and TC2. Table 1 shows the computed relative errors on measurements. They are below 1% for magnetics and 10% for interferometry and polarimetry which indicates an excellent convergence of the algorithm. Figure 5 shows the errors between reference and reconstructed profiles for the different cases and experiments. It appears clearly that this error is lower for MS than for MF and M. This is confirmed by Fig. 6 which shows the computed relative error in L^2 norm between the reconstructed and reference profiles. The use of Stokes vector measurements improves the identification of profiles A and B compared to the use of Faraday measurements and of magnetic measurements alone, particularly in the magnetic axis region.

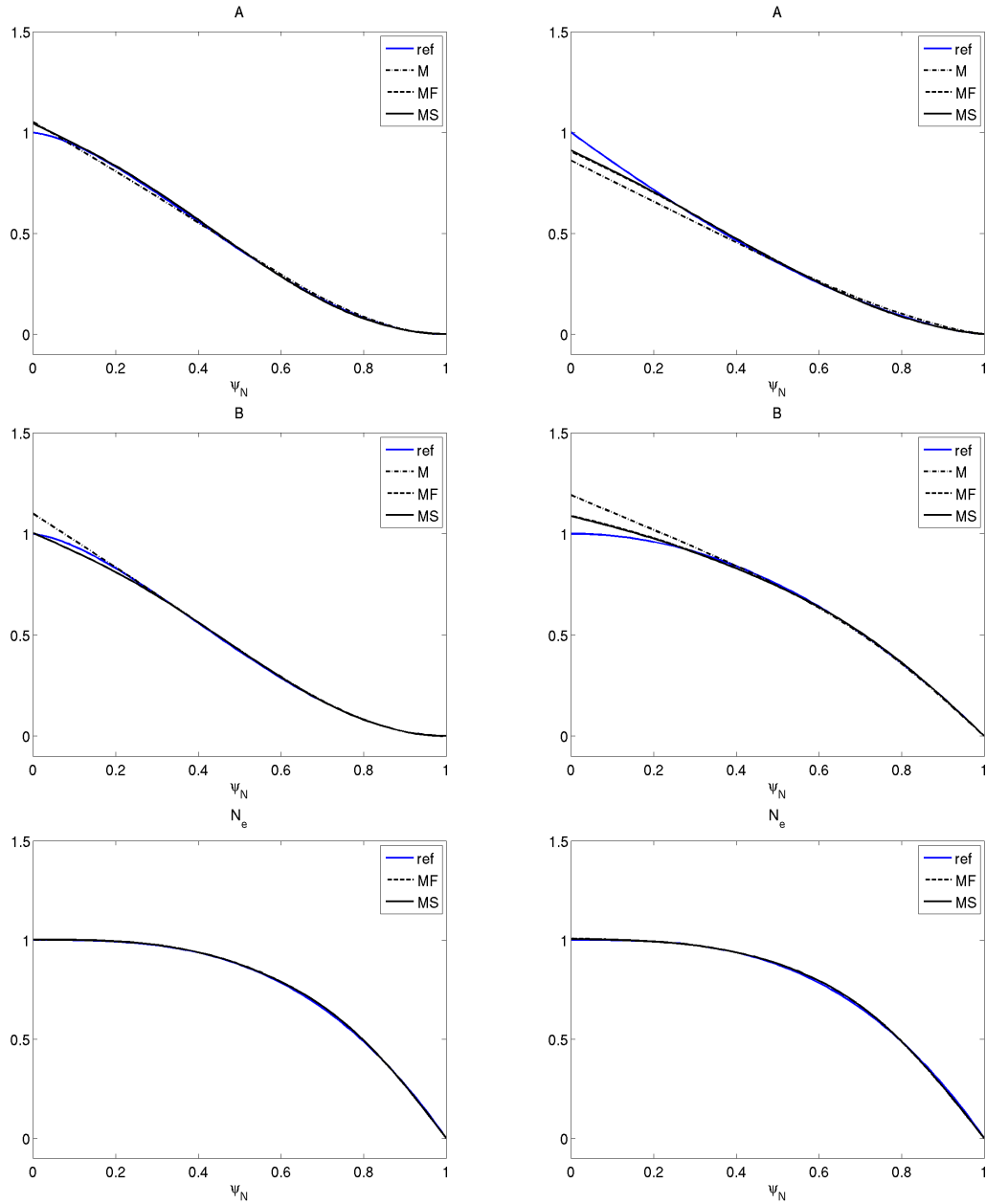


Figure 4: Reconstructed normalized A , B and N_e profiles for TC1 (left column) and TC2 (right column) for experiment M (magnetics only), MF (magnetics and Faraday) and MS (magnetics and Stokes). The reference profile is shown in blue.

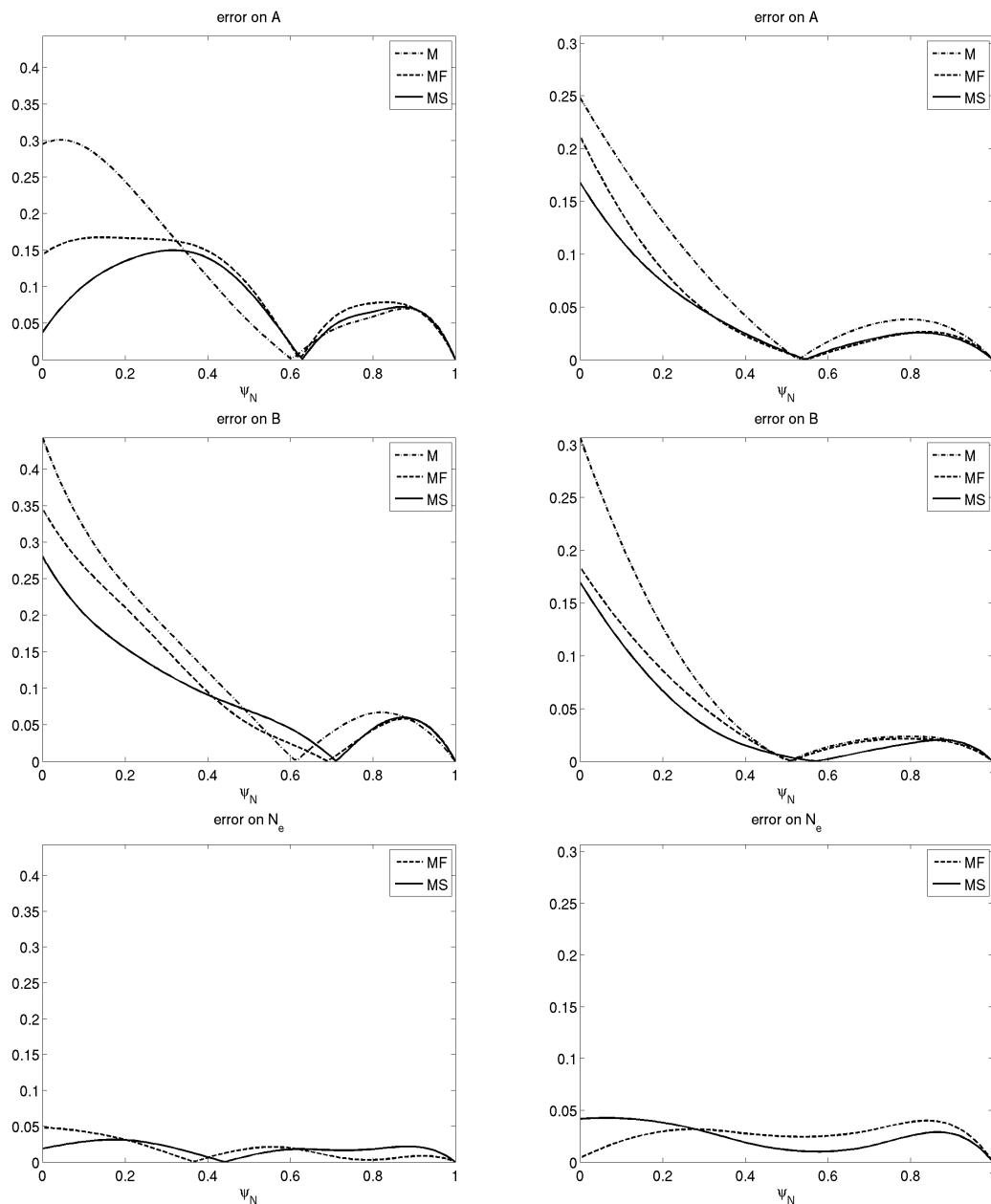


Figure 5: Normalized error between reconstructed and reference profile for A , B and N_e for experiments M, MF and MS (left TC1 and right TC2), with 1% noise on magnetics and 10% noise on interferometry and polarimetry.

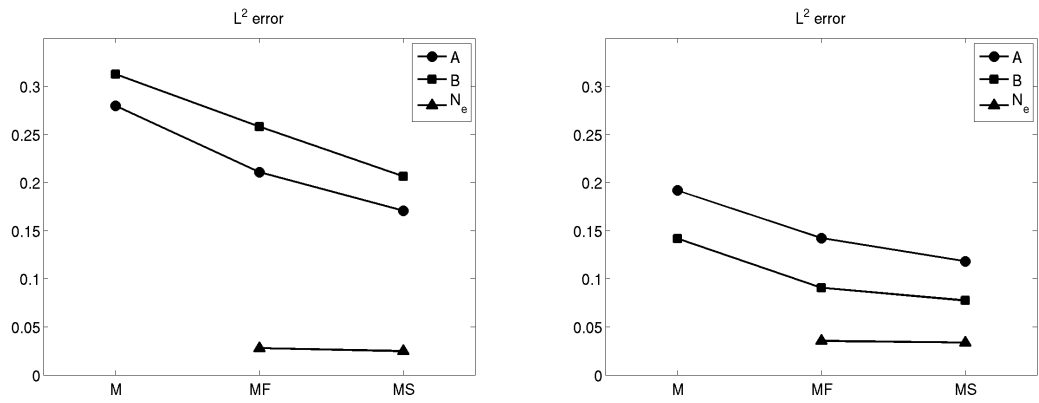


Figure 6: Relative error between reconstructed and reference profile for A , B and N_e in L^2 norm for experiments M, MF and MS, with 1% noise on magnetics and 10% noise on interferometry and polarimetry. Left: TC1. Right: TC2.

5 Conclusion

In this work is provided a numerical method enabling the consistent resolution of the inverse equilibrium reconstruction problem in the framework of non-linear free-boundary equilibrium coupled to the Stokes model equation for polarimetry. Using optimal control theory we derive the optimality system for this inverse problem in which the direct model consists in a non-linear partial differential equations and a set of systems of ordinary differential equation. An SQP method is proposed for the numerical resolution of the problem. Numerical experiments show that the method is efficient. However the complexity of the model considered here has lead us to formulate explicitly the adjoint model for the resolution of the inverse problem. This is expensive from the computational point of view and might not be suited for real-time reconstruction.

In addition the numerical experiments conducted in this study for an ITER configuration show that the accuracy of the identification of the unknown profile functions benefits from the use of Stokes vector measurements compared to the classical case of Faraday measurements. This is not suprising since more data are being use in the inverse resolution and the benefit should be even more interesting with real experimental measurements which are, according to the literature, more accurately represented by the Stokes model.

References

- [1] V. D. Shafranov, Determination of the parameters β_p and l_i in a tokamak for arbitrary shape of plasma pinch cross-section, *Plasma Physics* 13 (9) (1971) 757.
- [2] L. Zakharov, V. Shafranov, Equilibrium of a toroidal plasma with noncircular cross-section, *Sov. Phys. Tech. Phys.* 18 (2) (1973) 151–156.
- [3] J. Luxon, B. Brown, Magnetic analysis of non-circular cross-section tokamaks, *Nuclear Fusion* 22 (6) (1982) 813–821.
- [4] D. Swain, G. Neilson, An efficient technique for magnetic analysis for non-circular, high-beta tokamak equilibria, *Nuclear Fusion* 22 (8) (1982) 1015–1030.
- [5] L. Lao, Separation of β_p and l_i in tokamaks of non-circular cross-section, *Nuclear Fusion* 25 (11) (1985) 1421.
- [6] J. Blum, *Numerical Simulation and Optimal Control in Plasma Physics with Applications to Tokamaks*, Series in Modern Applied Mathematics, Wiley Gauthier-Villars, Paris, 1989.
- [7] H. Grad, H. Rubin, Hydromagnetic equilibria and force-free fields, *Proceedings of the 2nd UN Conf. on the Peaceful Uses of Atomic Energy* 31 (1958) 190.
URL http://www-naweb.iaea.org/napc/physics/2ndgenconf/data/Proceedings1958/papersVol131/Paper25_Vol131.pdf
- [8] V. Shafranov, On magnetohydrodynamical equilibrium configurations, *Soviet Journal of Experimental and Theoretical Physics* 6 (1958) 545.
URL <http://adsabs.harvard.edu/abs/1958JETP...6..545S>
- [9] R. Lüster, A. Schlüter, Axialsymmetrische magnetohydrodynamische Gleichgewichtskonfigurationen., *Z. Naturforsch. A* 12 (1957) 850–854.
- [10] L. Lao, J. Ferron, R. Geobner, W. Howl, H. St. John, E. Strait, T. Taylor, Equilibrium analysis of current profiles in Tokamaks, *Nuclear Fusion* 30 (6) (1990) 1035.
- [11] J. Blum, E. Lazzaro, J. O'Rourke, B. Keegan, Y. Stefan, Problems and methods of self-consistent reconstruction of tokamak equilibrium profiles from magnetic and polarimetric measurements, *Nuclear Fusion* 30 (8) (1990) 1475.
- [12] P. J. Mc Carthy, Analytical solutions to the Grad-Shafranov equation for tokamak equilibrium with dissimilar source functions, *Physics of Plasmas* 6 (9) (1999) 3554–3560.
doi:<http://dx.doi.org/10.1063/1.873630>
URL <http://scitation.aip.org/content/aip/journal/pop/6/9/10.1063/1.873630>
- [13] W. Zwingmann, Equilibrium analysis of steady state tokamak discharges, *Nuclear Fusion* 43 (2003) 842–850.
- [14] J. Blum, C. Boulbe, B. Faugeras, Real-time plasma equilibrium reconstruction in a tokamak, in: *Journal of Physics: Conference Series. Proceedings of the 6th International Conference on Inverse Problems in Engineering: Theory and Practice*, Vol. 135, IOP Publishing, 2008, p. 012019.

- [15] J. Blum, C. Boulbe, B. Faugas, Reconstruction of the equilibrium of the plasma in a tokamak and identification of the current density profile in real time, *Journal of Computational Physics* 231 (3) (2012) 960 – 980. doi:<http://dx.doi.org/10.1016/j.jcp.2011.04.005>. URL <http://www.sciencedirect.com/science/article/pii/S0021999111002294>
- [16] B. Faugas, J. Blum, C. Boulbe, P. Moreau, E. Nardon, 2D interpolation and extrapolation of discrete magnetic measurements with toroidal harmonics for equilibrium reconstruction in a Tokamak, *Plasma Phys. Control Fusion* 56 (2014) 114010.
- [17] J.-M. Moret, B. Duval, H. Le, S. Coda, F. Felici, H. Reimerdes, Tokamak equilibrium reconstruction code LIUQE and its real time implementation, *Fusion Eng. Design* 91 (0) (2015) 1–15. doi:<http://dx.doi.org/10.1016/j.fusengdes.2014.09.019>. URL <http://www.sciencedirect.com/science/article/pii/S0920379614005973>
- [18] V. Pustovitov, Magnetic diagnostics: General principles and the problem of reconstruction of plasma current and pressure profiles in toroidal systems, *Nuclear Fusion* 41 (6) (2001) 721.
- [19] F. Hofmann, G. Tonetti, Tokamak equilibrium reconstruction using Faraday rotation measurements, *Nuclear Fusion* 28 (10) (1988) 1871. URL http://iopscience.iop.org/0029-5515/28/10/014/pdf/0029-5515_28_10_014.pdf
- [20] J. Blum, H. Buvat, An inverse problem in plasma physics: the identification of the current density profile in a Tokamak, in: Biegler, Coleman, Conn, Santosa (Eds.), *IMA Volumes in Mathematics and its Applications, Volume 92, "Large Scale Optimization with applications"*, Part 1: Optimization in inverse problems an design, Springer, New York, 1997, pp. 17–36.
- [21] F. Orsitto, A. Boboc, C. Mazzotta, E. Giovannozzi, L. Zabeo, J. Contributors, Modelling of polarimetry measurements at JET, *Plasma Phys. Control Fusion* 50 (2008) 115009.
- [22] F. Orsitto, A. Boboc, P. Gaudio, M. Gelfusa, E. Giovannozzi, C. Mazzotta, A. Murari, J. Contributors, Analysis of faraday rotation in JET polarimetry measurements, *Plasma Phys. Control Fusion* 53 (2011) 035001.
- [23] S. E. Segre, A review of plasma polarimetry - theory and methods, *Plasma Phys. Control Fusion* 41 (2) (1999) R57. URL <http://stacks.iop.org/0741-3335/41/i=2/a=001>
- [24] R. Imazawa, Y. Kawano, Y. Kusama, A new approach of equilibrium reconstruction for ITER, *Nuclear Fusion* 51 (2011) 113022.
- [25] J. Chrzanowski, Y. Kravtsov, A new method of determining the parameters of thermonuclear plasma on the basis of multichannel polarimetric measurements, *Nukleonika* 58 (2) (2013) 281–285.
- [26] J. P. Freidberg, *Ideal Magnetohydrodynamics*, Plenum US, 1987.
- [27] J. Wesson, *Tokamaks*, The International Series of Monographs in Physics, Oxford University Press, 2004.
- [28] J. P. Goedbloed, S. Poedts, *Principles of magnetohydrodynamics: with applications to laboratory and astrophysical plasmas*, Cambridge university press, 2004.

- [29] S. Jardin, Computational methods in plasma physics, Boca Raton, FL : CRC Press/Taylor & Francis, 2010.
- [30] H. Heumann, J. Blum, C. Boulbe, B. Faugeras, G. Selig, J.-M. Ané, S. Brémont, V. Grandgirard, P. Hertout, E. Nardon, Quasi-static free-boundary equilibrium of toroidal plasma with CEDRES++: computational methods and applications, *J. Plasma Physics* doi:<http://dx.doi.org/10.1017/S0022377814001251>.
- [31] R. Albanese, J. Blum, O. Barbieri, On the solution of the magnetic flux equation in an infinite domain, in: EPS. 8th Europhysics Conference on Computing in Plasma Physics (1986), 1986, pp. 41–44.
- [32] G. Gatica, G. Hsiao, The uncoupling of boundary integral and finite element methods for nonlinear boundary value problems, *J. Math. Anal. Appl.* 189 (2) (1995) 442–461. doi:10.1006/jmaa.1995.1029. URL <http://dx.doi.org/10.1006/jmaa.1995.1029>
- [33] V. Grandgirard, Modélisation de l'équilibre d'un plasma de tokamak, Ph.D. thesis, Université de Franche-Comté (1999).
- [34] J. Blum, J. Le Foll, B. Thooris, The self-consistent equilibrium and diffusion code SCED, *Computer Physics Communications* 24 (1981) 235 – 254. doi:10.1016/0010-4655(81)90149-1. URL <http://www.sciencedirect.com/science/article/pii/0010465581901491>
- [35] R. Albanese, J. Blum, O. De Barbieri, Numerical studies of the Next European Torus via the PROTEUS code, in: 12th Conf. on Numerical Simulation of Plasmas, San Francisco, 1987.
- [36] M. Hinze, R. Pinnau, M. Ulbrich, S. Ulbrich, Optimization with PDE constraints, Vol. 23 of *Mathematical Modelling: Theory and Applications*, Springer, New York, 2009.
- [37] J. Nocedal, S. J. Wright, Numerical optimization, 2nd Edition, Springer Series in Operations Research and Financial Engineering, Springer, New York, 2006.
- [38] ITM, Integrated Tokamak Modelling, <http://portal.efda-itm.eu/> (2013).
- [39] G. Falchetto, D. Coster, R. Coelho, B. Scott, L. Figini, D. Kalupin, E. Nardon, S. Nowak, L. Alves, J. Artaud, V. Basiuk, J. P. Bizarro, C. Boulbe, A. Dinklage, D. Farina, B. Faugeras, J. Ferreira, A. Figueiredo, P. Huynh, F. Imbeaux, I. Ivanova-Stanik, T. Jonsson, H.-J. Klingshirn, C. Konz, A. Kus, N. Marushchenko, G. Pereverzev, M. Owsiak, E. Poli, Y. Peysson, R. Reimer, J. Signoret, O. Sauter, R. Stankiewicz, P. Strand, I. Voitsekhovitch, E. Westerhof, T. Zok, W. Zwingmann, I.-T. Contributors, the ASDEX Upgrade Team, J.-E. Contributors, The European Integrated Tokamak Modelling (ITM) effort: achievements and first physics results, *Nuclear Fusion* 54 (4) (2014) 043018. URL <http://stacks.iop.org/0029-5515/54/i=4/a=043018>
- [40] T. Yamaguchi, Y. Kawano, H. Fujieda, K. Kurihara, M. Sugihara, Y. Kusama, Optimization of the viewing chord arrangement of the ITER poloidal polarimeter, *Plasma Phys. Control Fusion* 50 (2008) 045004.



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ISSN 0249-6399